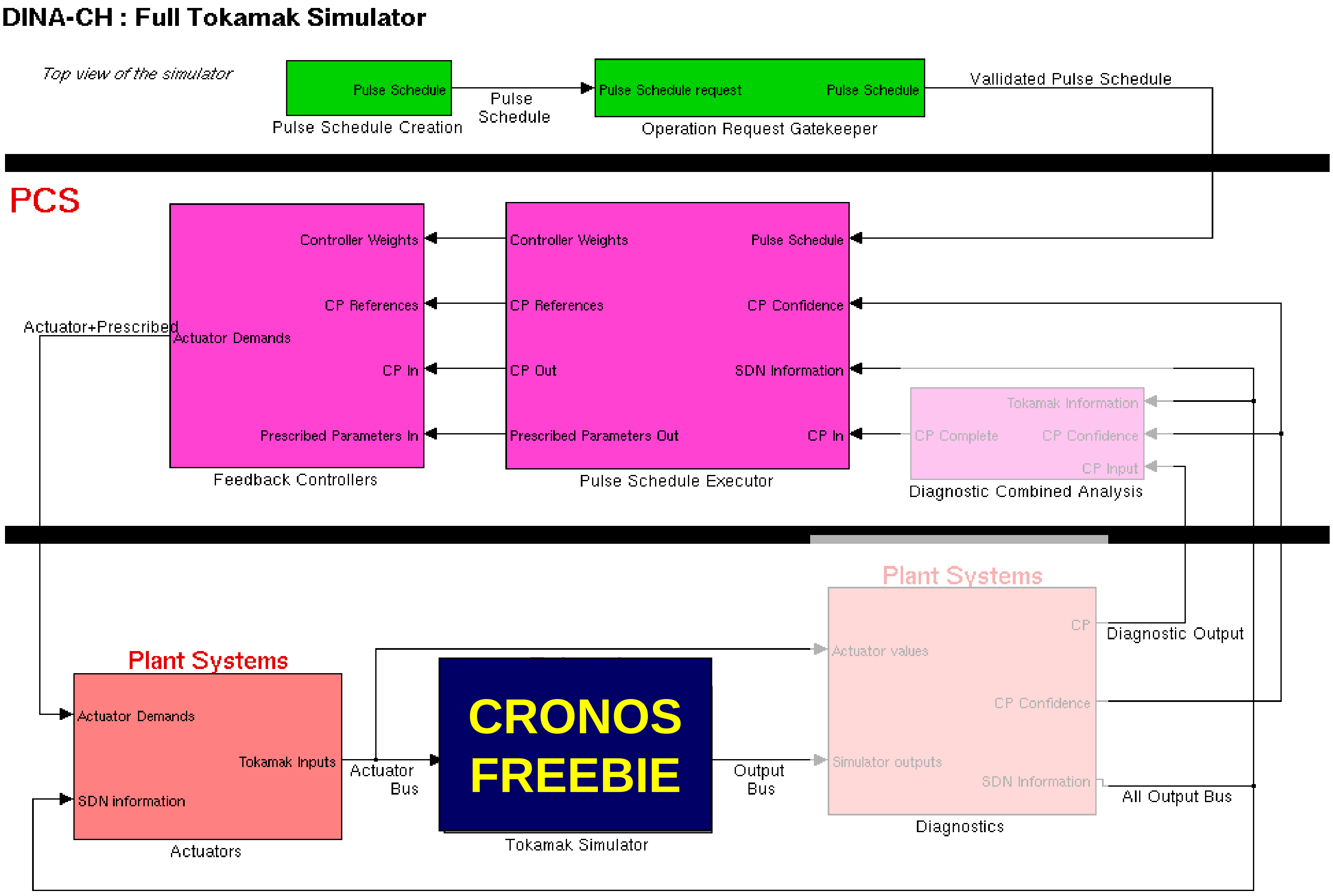
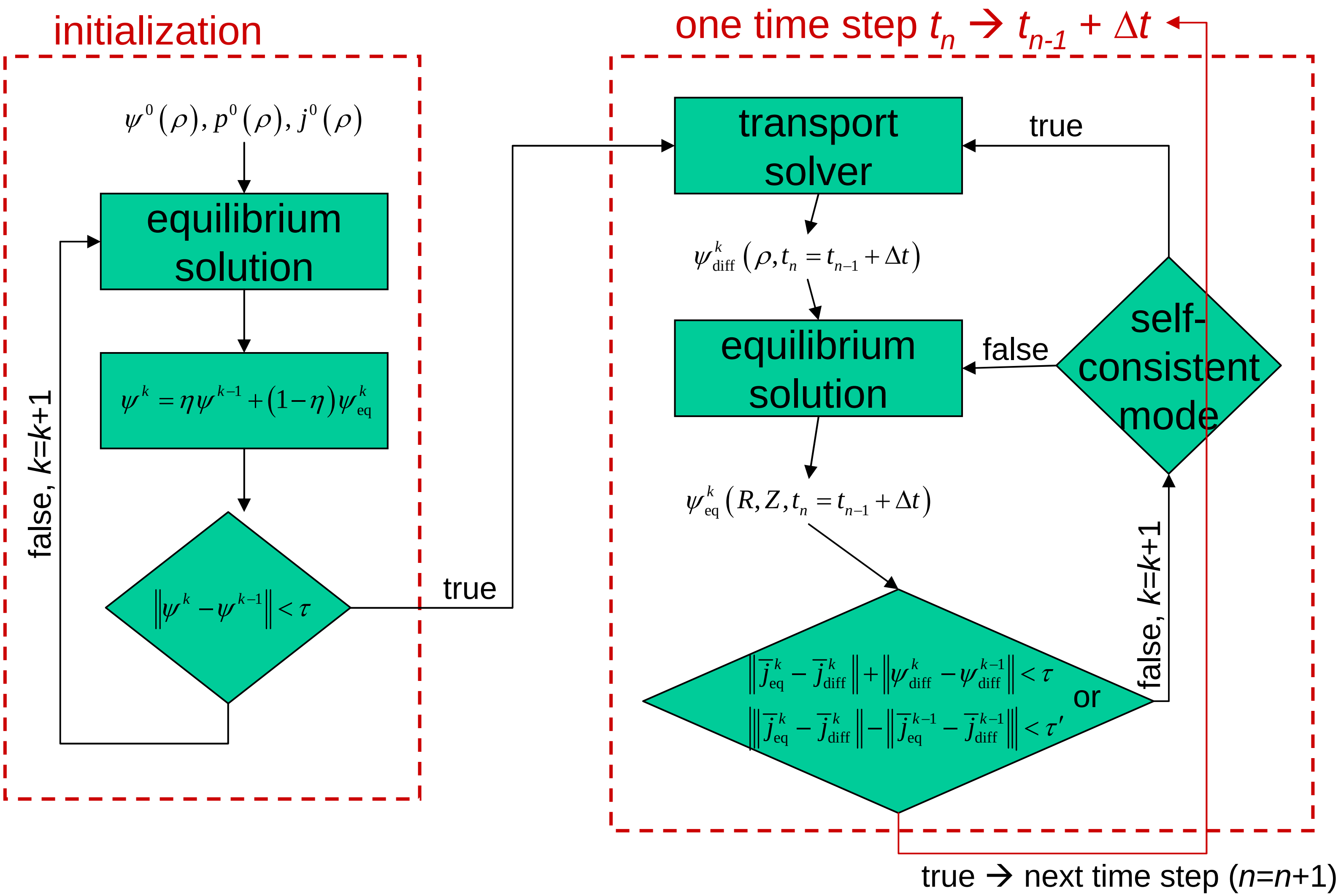
Free-boundary equilibrium transport simulations of ITER scenarios under control

J. Urban[1,2], J.F. Artaud[1], K. Besseghir[3], J.B. Lister[3], J. Garcia[1], F. Imbeaux[1], S.H. Kim[4], E. Nardon[1], R. Nouailletas[1]

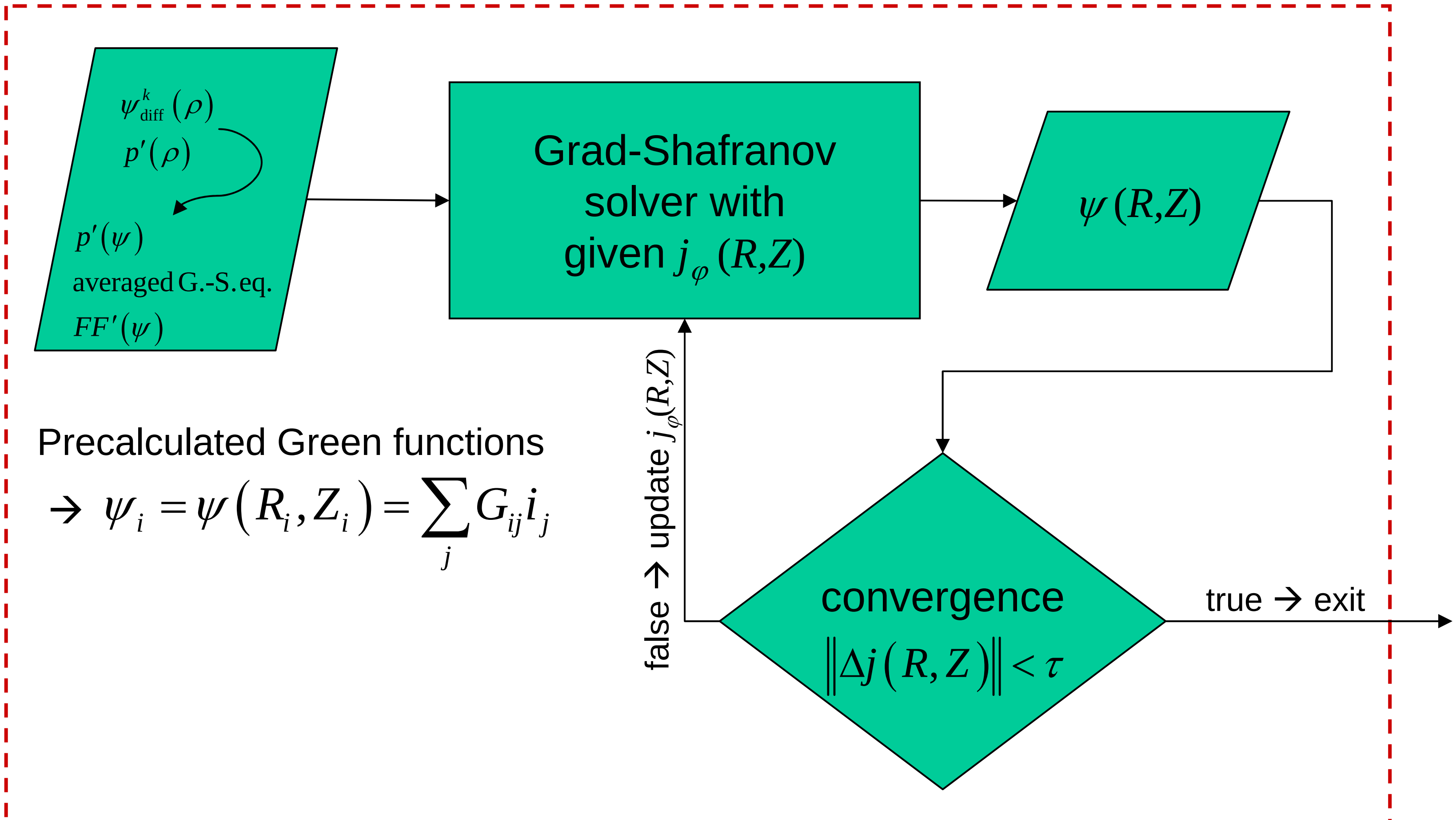
Self-consistently coupled CRONOS/FREEBIE is integrated in the full tokamak simulator framework of CRPP



Equilibrium-transport convergence is assured every time step



The equilibrium convergence loop



Precalculated Green functions

$$\rightarrow \psi_i = \psi(R_i, Z_i) = \sum_j G_{ij} i_j$$

Dirichlet current diffusion boundary condition (predictor-corrector)

$$\psi_{\mathrm{a}} - \psi_{\mathrm{b}} = L_{\mathrm{p}} I_{\mathrm{p}}$$

L_{p} contains contribution from plasma and coils (equilibrium coil currents ~ I_{p}).

$$\Delta\psi = \psi_{\mathrm{b}}^{\mathrm{diff}} - \psi_{\mathrm{b}}^{\mathrm{equil}}$$

$$I_{\mathrm{p}}^{*} = I_{\mathrm{p}}\left(1 + \frac{\Delta\psi}{\psi_{\mathrm{a}} - \psi_{\mathrm{b}}}\right)$$

Use I_p^* as next time step boundary condition for the current diffusion equation.

Robin boundary conditions

$$\psi_{\mathrm{a}} = \psi_{\mathrm{a}}^{\mathrm{ext}} + \psi_{\mathrm{a}}^{\mathrm{pl}}$$

Separate the flux at the axis into plasma and external part (straightforward in FREEBIE).

$$\psi_{\mathrm{b}} = \psi_{\mathrm{a}}^{\mathrm{ext}} + L_{\mathrm{ext}} I_{\mathrm{p}}$$

Again, make use of inductances.

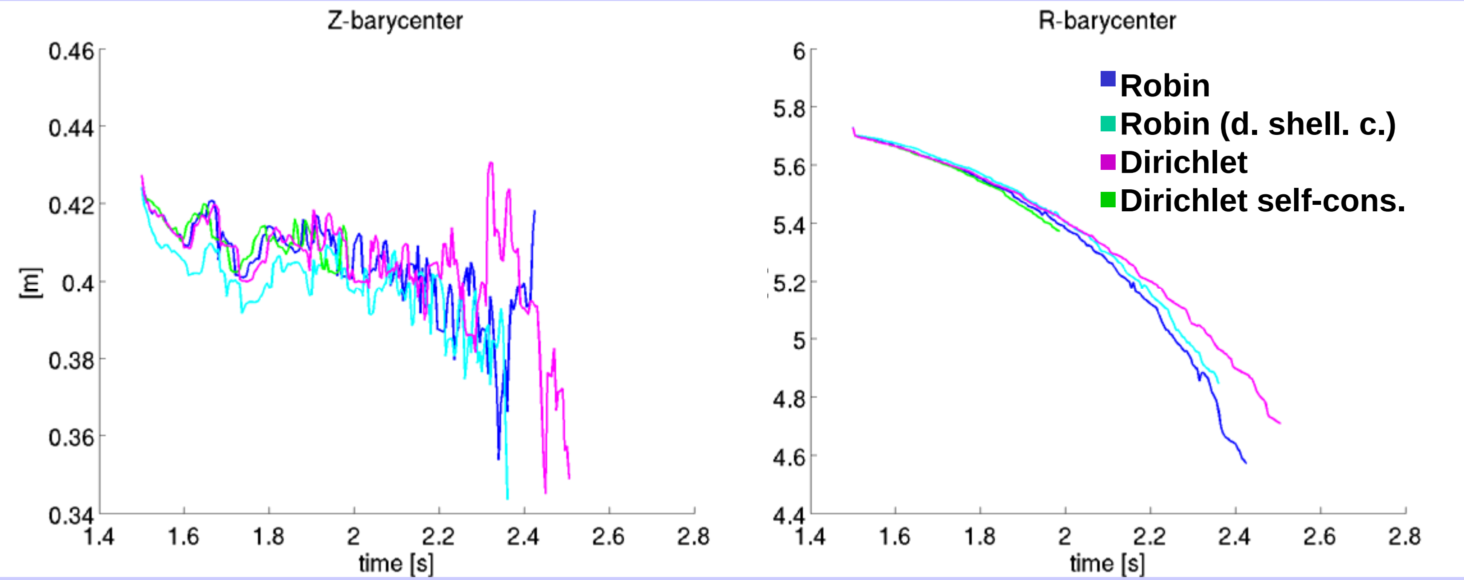
Discrete time derivative finally yields a Robin boundary condition for ψ_{b}.

$$\frac{\psi_{\mathrm{b}}}{\tilde{L}_{\mathrm{ext}}} - \gamma C \left.\frac{\partial \psi}{\partial \rho}\right|_{\rho=1} = \frac{1}{\tilde{L}_{\mathrm{ext}}}\left(\tilde{\psi}_{\mathrm{b}} + \psi_{\mathrm{a}}^{\mathrm{ext}} - \tilde{\psi}_{\mathrm{a}}^{\mathrm{ext}}\right) + \tilde{I}_{\mathrm{p}}$$

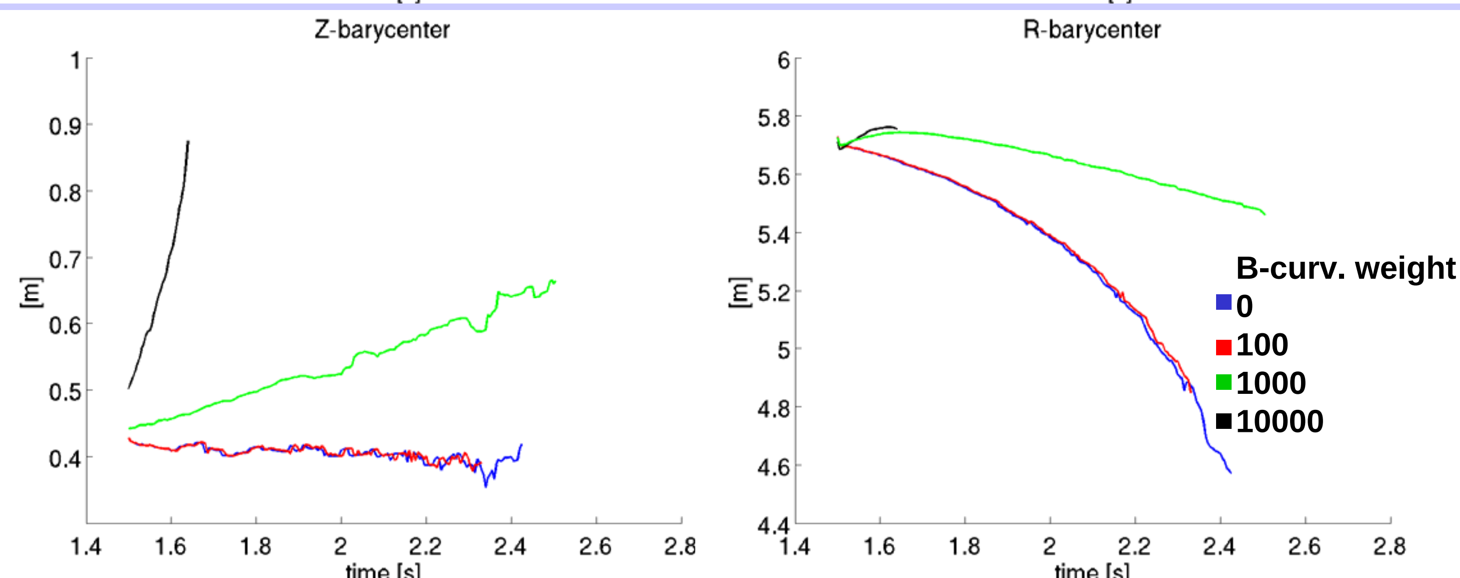
Tilde is for previous time step values.

$$\gamma = L_{\mathrm{ext}} \,/\, \tilde{L}_{\mathrm{ext}}$$

Circular plasma is vertically stable for a wide range of parameters

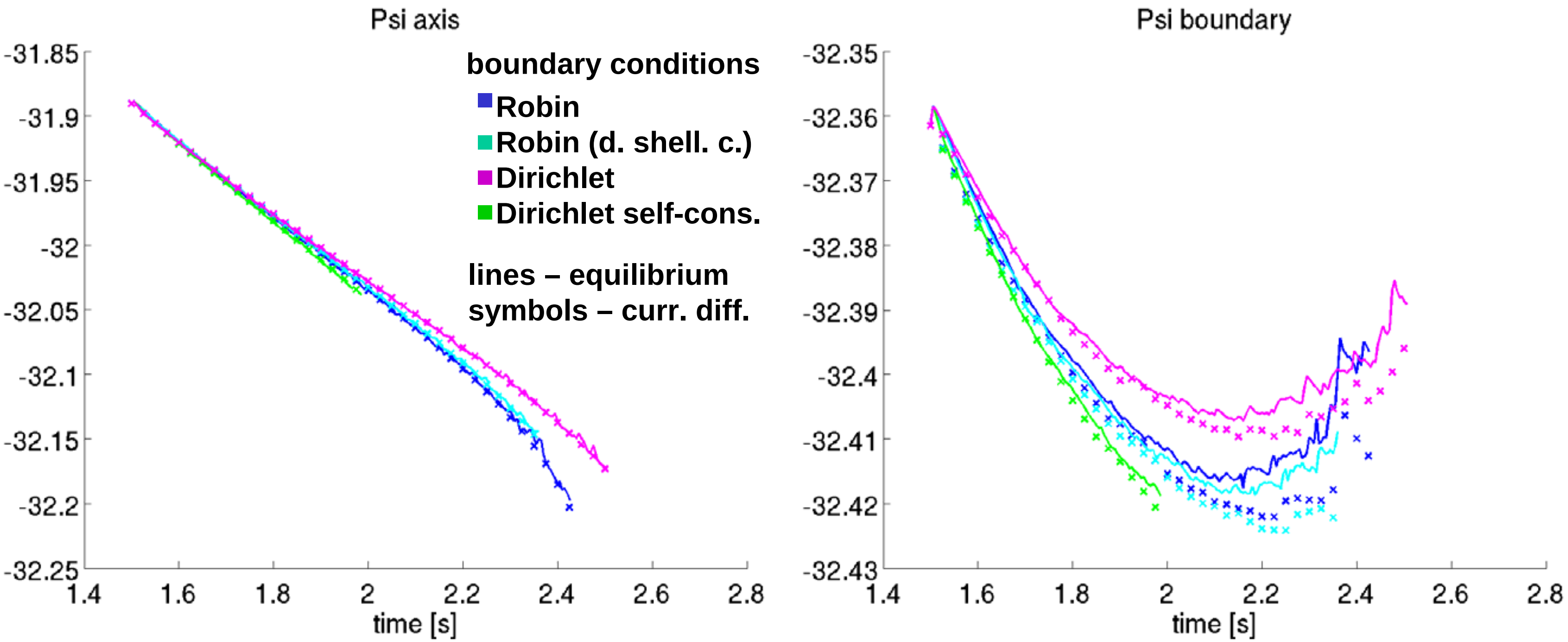


← Almost the same results for different boundary conditions. Self-consistent mode stopped converging sooner.



← B-curvature constraint in the initial inverse mode can lead to vertically unstable plasmas.

Psi is consistent on the axis and the boundary



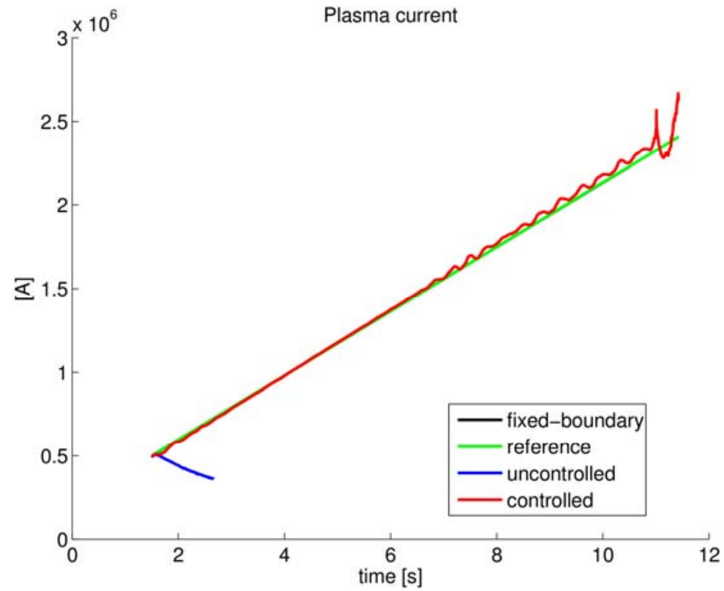
- Psi consistency is not automatic
  - convergence criterion is on $<j>$.
  - normalization by I_p or ψ_a-ψ_b

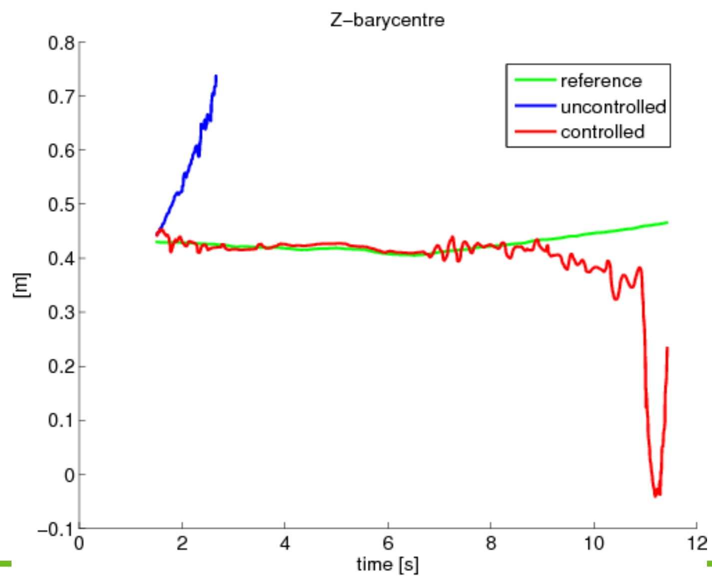
Feed-forward currents (shaping) are calculated by CRONOS/FREEBIE

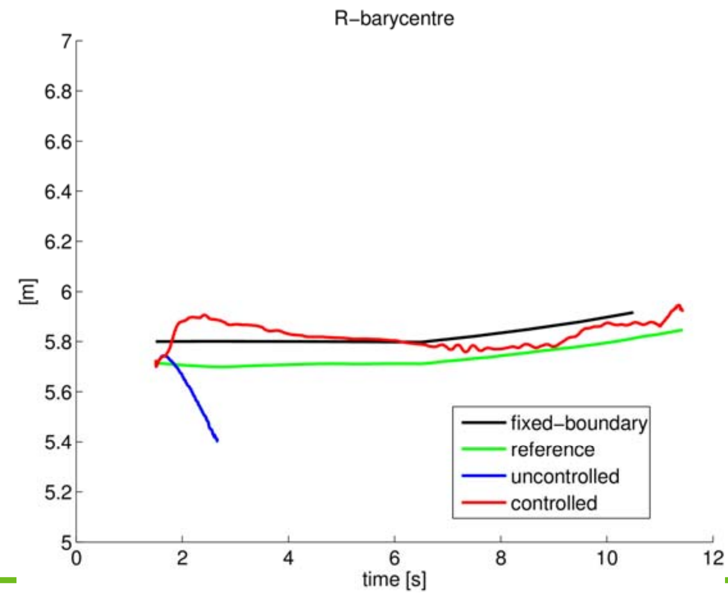
- During the initial phase of the discharge, only the position and I_p is controlled ➔ feed-forward currents are needed for plasma shaping.
  - These currents can be calculated by FREEBIE in inverse or Poynting mode.
  - Later on, gaps control is turned on (too slow for the ramp-up).
- Some coils are saturated for the test case.
  - Need to change the scenario?

Example – initial phase of the ITER hybrid scenario under control

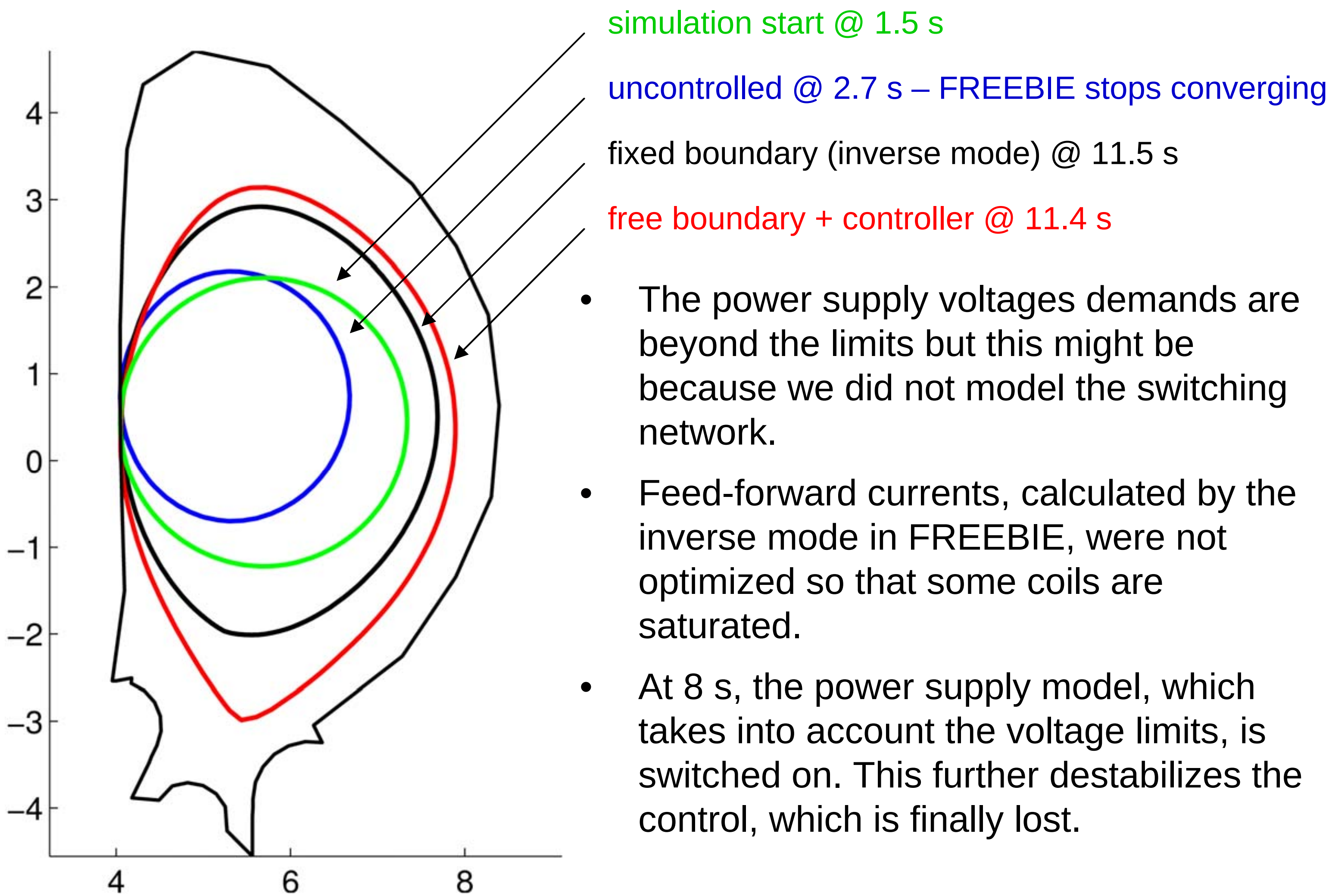
- The coupling became operational only recently.
- Plasma follows reasonably the waveforms.
- Optimizations of the initial equilibrium stability and target waveforms are ongoing.
  - Saturated coil currents from the inverse mode.





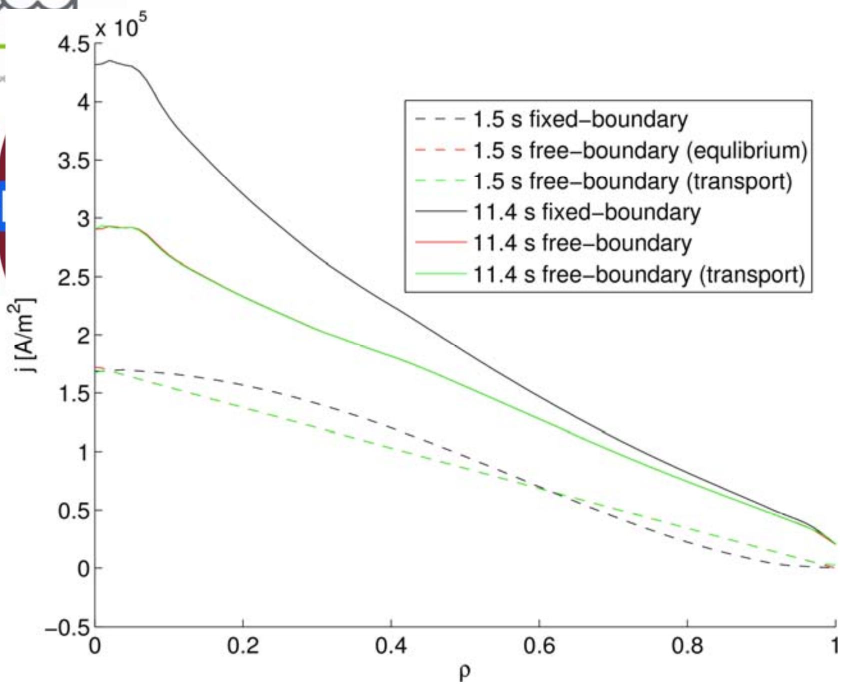


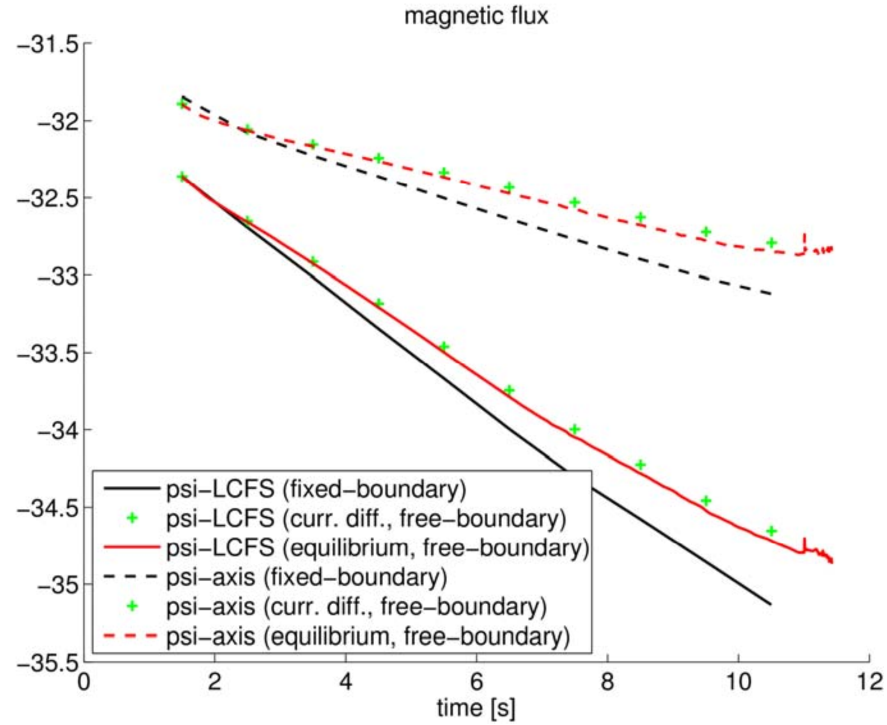
Plasma shaping working but not ideal



- The power supply voltages demands are beyond the limits but this might be because we did not model the switching network.
- Feed-forward currents, calculated by the inverse mode in FREEBIE, were not optimized so that some coils are saturated.
- At 8 s, the power supply model, which takes into account the voltage limits, is switched on. This further destabilizes the control, which is finally lost.

Magnetic fluxes and current densities are consistent with current diffusion

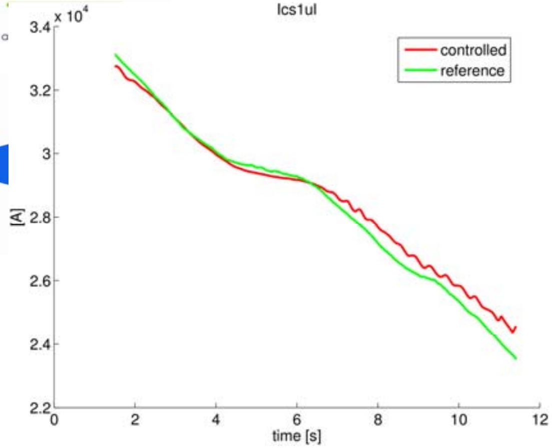


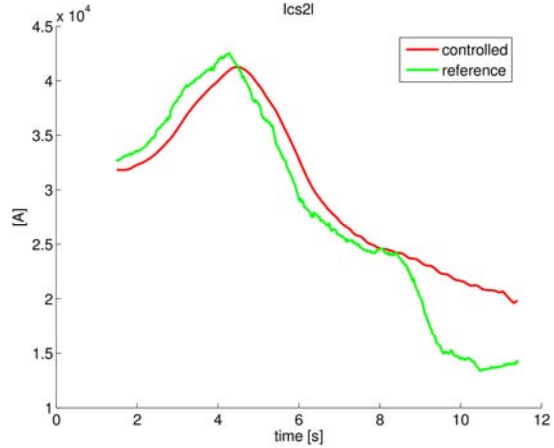


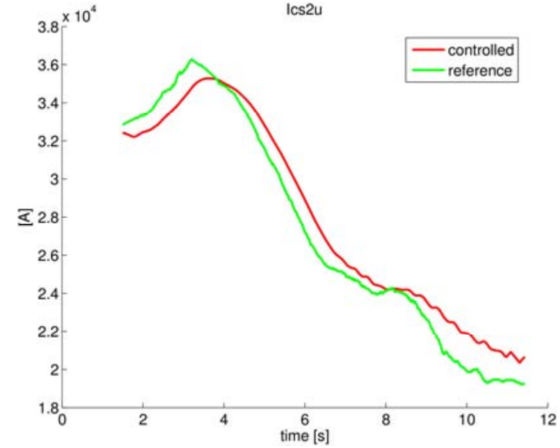
Averaged plasma current density comparison at 1.5 s and at 11.4 s. The difference between fixed and free-boundary solution is mainly caused by the very different plasma size.
Note that the equilibrium and current diffusion profiles are perfectly consistent (red and green superimposed in the figure).

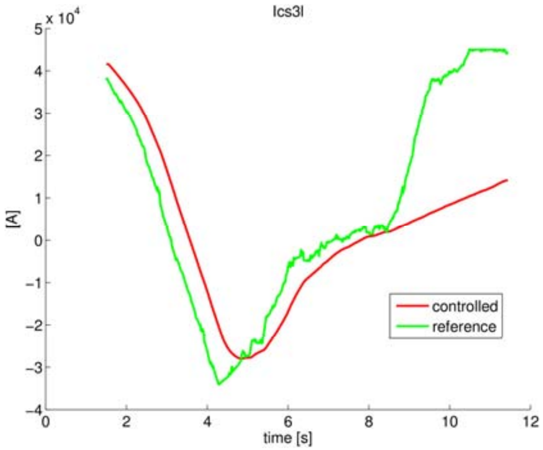
Magnetic flux evolution at the magnetic axis and at the plasma boundary. Comparison with the current diffusion (transport) values.

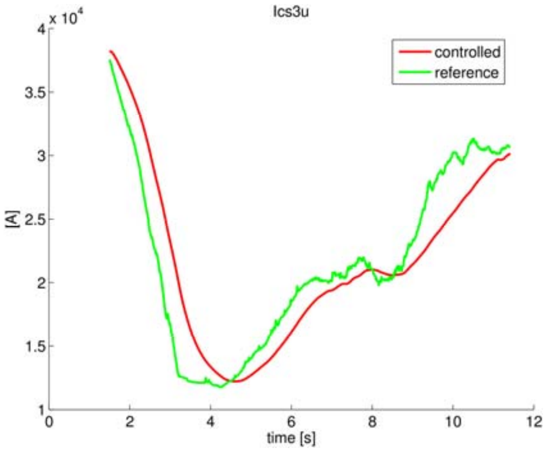
CS currents follow the references unless voltage limits are exceeded after 8 s



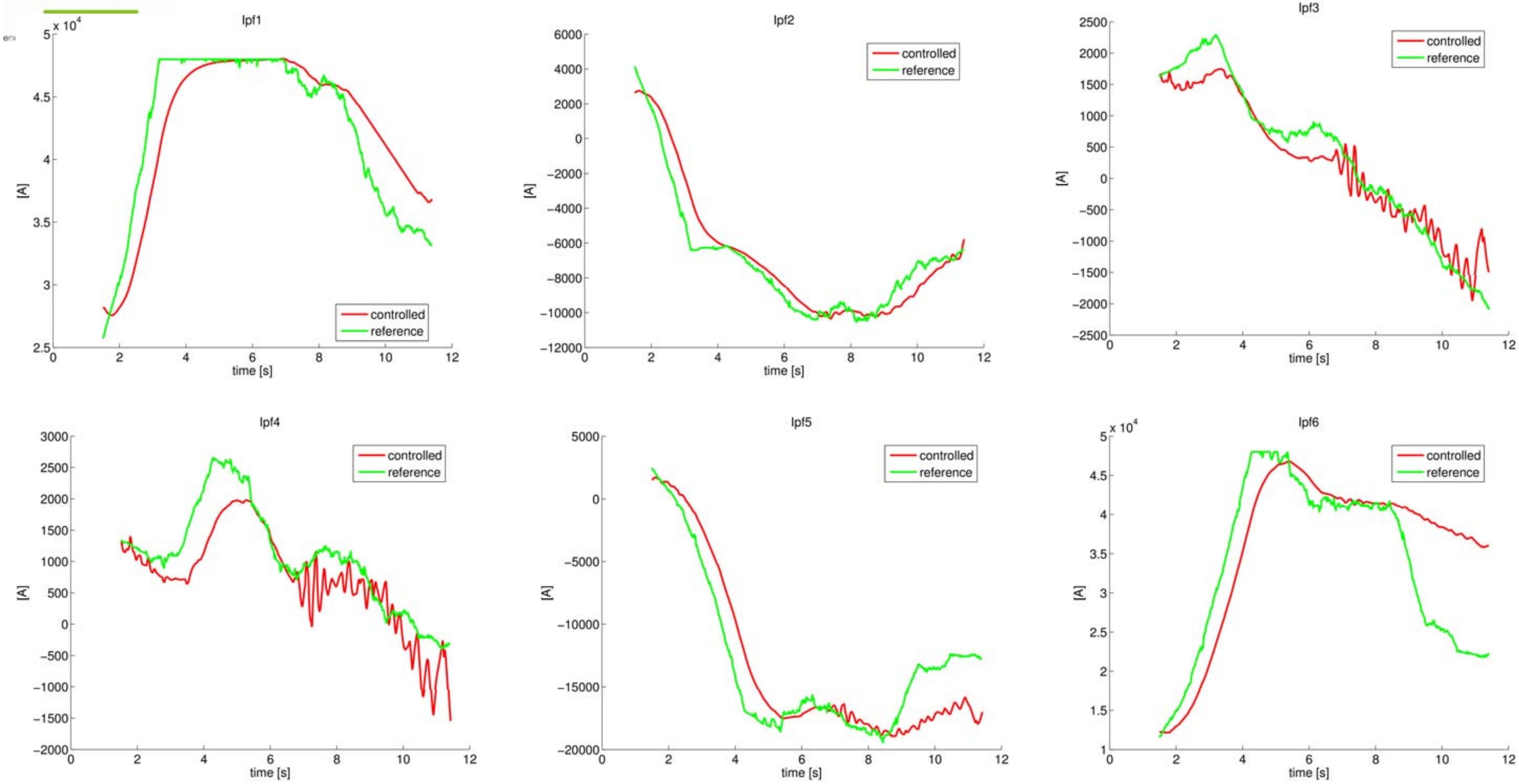




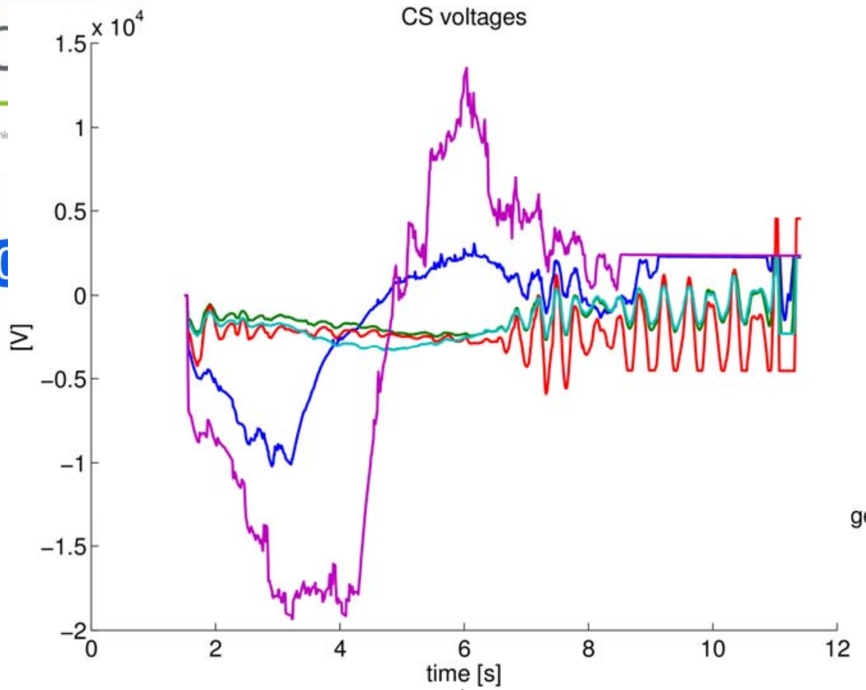


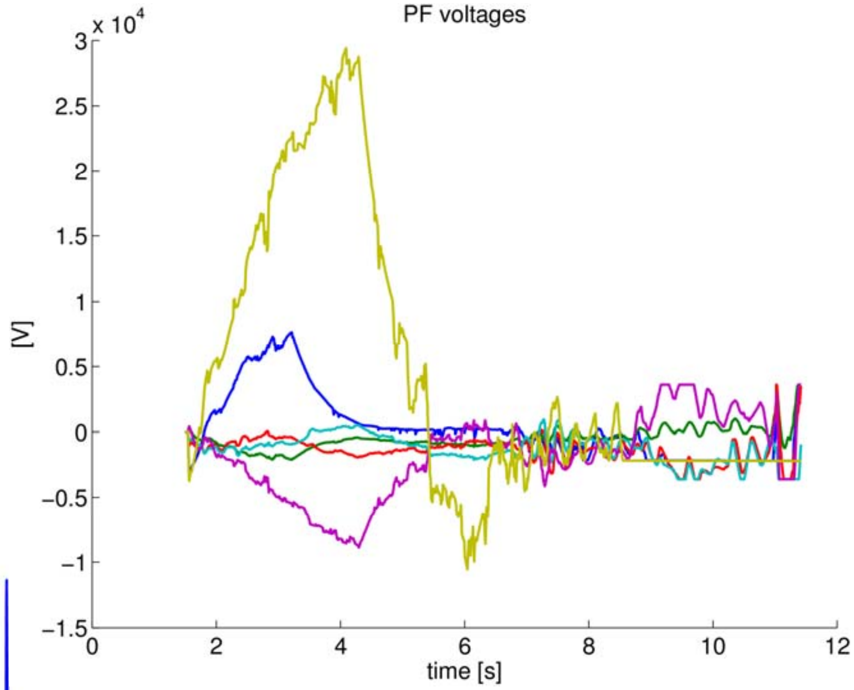


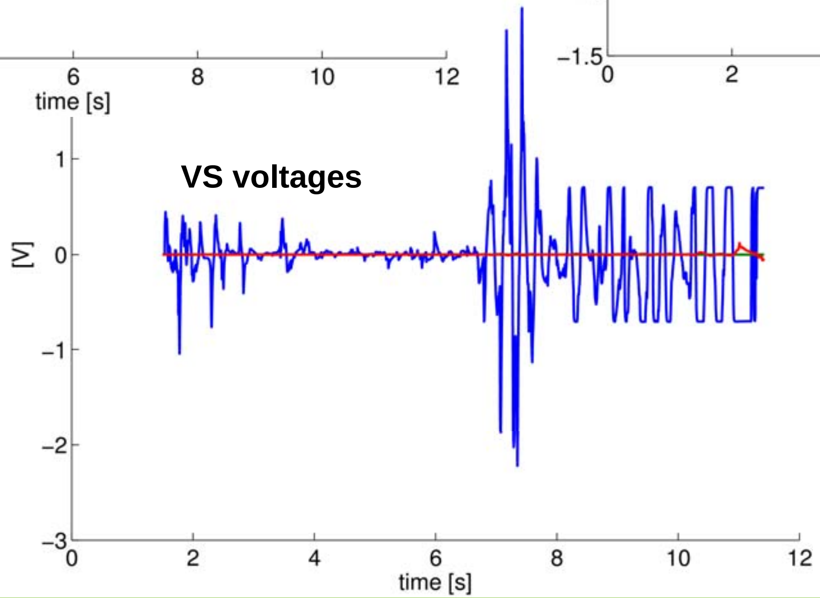
Some PF coils are saturated in inverse mode



PF supplies voltages exceed the limits







- Initial plasma stability
- Induced currents evolution
  - Huge currents growth in the cryostat – had to be switched off
  - Convergence problems when shell currents are neglected
- Mesh resolution for flat-top, fully shaped plasma
  - Deformed boundary in inverse mode
- Target optimization (feed-forward)
  - shape, current ramp, coil currents

What to focus on in future?

- Constraints from PF power supplies (voltage + currents) and PF coils geometry
- Optimizations
  - shape (currents demands, stability)
  - current ramp up/down
  - the right figure of merit?
- What are the major effects of equilibrium on transport and stability?