EUROPEAN FUSION DEVELOPMENT AGREEMENT

The European Integrated Tokamak Modelling Effort: Achievements and First Physics Results

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The Integrated Tokamak Modelling Task Force (ITM-TF) [1,2] was established in 2004 to coordinate the European tokamak modelling activities under EFDA. AIM: provide a standardized simulation framework supplying transparent, consistent and efficient integration of most of the state-of-the-art EU codes modelling both the plasma physics and the tokamak subsystems, for the simulation of present experiments and the prediction of complete discharges in ITER and beyond.

EU ITM-TF philosophy and approach

Comprehensive integrated tokamak modelling:

- infrastructure describing both the tokamak physics and the machine within a unique framework
- strategy: divide the global problem into Elementary Physics Problems (equilibrium, transport, MHD, sources, diagnostic response, ...)
- fully modular and flexible simulation platform
- standardized interfaces for physics and technology
 generic data and communication ontology
 - Consistent Physical Objects (CPO) [3]
- completely generic workflow

Equilibrium reconstruction and MHD stability chain

The ITM-TF integrated workflow coupling a free-boundary equilibrium reconstruction code, EQUAL [8](or CLISTE, CEDRES++...), a high resolution fixed-boundary Grad-Shafranov solver, HELENA, and a linear MHD stability code, ILSA [7, refs therein] was used to carry out first physical studies:

✓ analysis of edge MHD stability of ASDEX Upgrade type-I ELMy

The EU ITM-TF framework is a valuable environment for both code benchmarking and coupling into complex integrated simulations: different physics codes are integrated as modules sharing the same standardized interface for data input/output and can be easily interchanged in the same ITM workflow
> minimize the possible sources of discrepancy in benchmarks
> easily refine the physics description

Benchmarking of electron cyclotron heating and current drive codes on an ITER scenario

A benchmark among five European EC beam/ray-tracing codes (C3PO, GRAY, TORAY-FOM, TORBEAM, TRAVIS)[9,refs therein] was successfully performed for the standard inductive H-mode ITER "Scenario 2" for three different launching conditions from the Equatorial Launcher (EL) and Upper Launcher (UL).

Divergent beam from EL at small toroidal launching angle → Heating and CD in the core
 Same as 1), with larger toroidal launching angle → off-axis H&CD

3) Focused beam from the UL, aimed at the q=3/2 flux surface

→ (Antennas CPO)

Case $R_0(m) z_0(m) \alpha (\circ) \beta (\circ) w_0(m) d(m)$ For all the cases:

(R₀,z₀): launching point

H-mode discharges

\checkmark core and pedestal scans of β_{N} for an ITER hybrid scenario





\succ peaking positions match within $\delta \rho \sim 0.02$



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** See the Appendix of F. Romanelli et al., Proceedings of the 24th IAEA Fusion Energy Conference 2012, San Diego, US.



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Core-edge coupling within EU-ITM infrastructure

Automated **direct coupling** of an edge 2D transport code (SOLPS) [11]) and a 1D core transport code (ETS, the European Transport Solver [12], including an impurity module [4]) was demonstrated for the particular case of steady state and multiple impurities (D+He+C+Ar+Ne 42 states) [5]. Neutrals here not treated by the core codes.



Visualization of core-edge coupled simulation results using EU ITM-TF tools: electron temperature, calculated in the • core from ETS edge from SOLPS within defeatured 3D first wall of ASDEX Upgrade obtained by ray-tracing rasterization and smoothing [13]



Integration & validation of Synthetic Diagnostics

Turbulence-transport simulations

Neutral Particle Analyser (born neutrals from ASCOT + pitch velocity in collimator domain)



A generic framework for neutron synthetic diagnostics has been integrated (domain voxelization + neutron source rate/spectra + diagnostic response)





[6]

Synthetic reflectometry: full-wave 3D code (ERC3d) valid for both O and X-mode polarizations is being tested



Turbulence simulations for **JET hybrid discharge #77922** were performed with the electromagnetic non-linear gyrofluid GEM code [10] executed in batch on the HPC-FF within an ITM workflow. Experimental profiles at 47.7s (T_{e,i}, n_{e,i}, toroidal current) and basic MHD equilibrium geometry were given and s- α model assumed. GEM actor runs in parallel 8 independent fluxtube cases, at given reference points on the profile producing particle and heat fluxes profiles for both species. The experimental case was found to be ITG **stable** for most of the profile except for the edge point at r/a=0.96.



Validation of a spectral MSE forward model [14] for MSE spectra and charge exchange of the plasma + beam (full, $\frac{1}{2}$ and $\frac{1}{3}$ beam components included) started on ASDEX Upgrade.

> MSE emissivity wavelength spectra for ASDEX Upgrade shot #26323



A transport workflow was then setup: a fluxtube chain consisting of an independent run of the local delta-f model at each of 8 reference points on the profile, as above, but with the fluxes equalised using a profile modification algorithm designed to adjust the profiles into transport equilibrium (prescribed power profile with the correct total power at the outermost surface).

Each workflow step takes the plasma profiles as input, calculates an equilibrium_CPO using, at present, a shifted circular model, both CPOs are then input to GEM for a run segment of 10 τ_{GB} producing flux profiles, finally the simple equilibration model is used to adjust the temperature profiles.

Conventional direct comparison of turbulence simulations to experimental measurements and transport is often discrepant, unless some form of artificial convergence is used.

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A relaxation time constant is used, which needs to be calibrated. The workflow is being tested 2000 loop steps are used, corresponding to 2 10⁴ τ_{GB} but a fully relaxed case is not yet available.

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